Naturally Occurring Radionuclides in Sludge Samples from Some Egyptian Drinking Water Purification Stations

M. A. M. Uosif, Mahmoud Tammam, Shams A. M. Issa and Reda Elsaman

Physics Department, Faculty of Sciences, Al-Azhar University (Assiut branch), Egypt *Corresponding author: dr_mohamed_amin@lycos.com

Abstract

Sludge samples from nine drinking water purification stations in El-Mynia governorate namely Kedwan, Elhawarta, Demsher, Tewa, Ard Sultan, Tokh Elkhel, Abu Flow, Saft Elkhmar and Zohrt Elporgi, were analyzed for its naturally occurring radionuclides (226Ra, 232Th and 40K) by gamma- ray spectrometry system using (sodium iodide NaI (Tl) detector). The results show that the specific activity for 226Ra, 232Th and 40K in range of 6 ± 0.7 to 113 ± 6.7 , from 5 ± 1 to 117 ± 5.8 and from 47 ± 4.5 to 412 ± 20.6 Bq Kg-1 respectively. The results indicate that sludge of drinking water purification stations in El-Mynia governorate from large scale do not pose a significant radiological risk. Additionally evaluations have been made of the absorbed gamma dose rate in air and the annual effective dose equivalent.

Keywords: Natural radionuclides, drinking water purification stations and sediment

1. Introduction

Radiation in the environment from natural sources is the major source of radiation exposure to man. Radiation exposure results from the naturally-occurring radionuclides in the environment (terrestrial radiation) and direct cosmic (extra-terrestrial) radiation. Some sources of natural radiation have been enhanced (concentrated) by human technological activities and include wastes from mineral ores and the petroleum industry, sludge and scale from drinking water treatment.

The water has an importance in environmental studies because of its daily use for human consumption and its ability to transport pollutants. Sludge from drinking water treatment may contain both naturally-occurring and man-made radioactive materials. Water that originates in or moves through geologic deposits containing naturally-occurring radionuclides could result in radioactivity being carried to the treatment facility with storm water runoff or infiltration entering the sewer system, and water treatment plant residuals discharged to the sewer system.

The occurrence of natural radionuclides in drinking water poses a problem of health hazard, when these radionuclides are taken to the body by ingestion. Radionuclides in drinking water causes human internal exposure, caused by the decay of radionuclides taken into the body through ingestion and inhalation indirectly when they are incorporated as part of the human food chain [1]. In fact, the incoming water treated in these plants can contain such radionuclides as radium or uranium due to the geological media in which the waters flow. When this water is treated it passes through various filters to remove the contaminants. This

treatment may lead to the generation of radioactive wastes such as sludge samples or also to the radiological contamination of the filters used [2]. Surface water treatment processes included sedimentation, coagulation, flocculation and filtration, while the groundwater was treated using methods similar to surface water treatment [3].

Only radionuclides with half-lives comparable with the age of the earth or their corresponding decay products existing in terrestrial material such as 232Th, 238U and 40K are of great interest. Another minor source is the radioactive decay of the 235U isotope, but this is very rate in the earth's crust (only 0.72% compared with 99.27% of 238U of its total uranium content [4]. Gamma radiation from these represents the main external source of irradiation to the human body.

Natural gamma- ray spectroscopic analysis has several advantages: the gamma rays are capable of penetrating a few inches of rock or soil samples, and both NaI (TI) (with high efficiency) and HPGe detectors (with high-energy resolution) can be utilized for the analysis [5]. 238U and its daughter's rather than 226R and its daughter products are responsible for a major fraction of the internal dose received by humans from naturally occurring radionuclides [6].

The United Nations Scientific Committee on the Effects of Atomic Radiation (UNSCEAR), has estimated that exposure to natural sources contributes >70% of the population radiation dose and the global average human exposure from natural sources is 2.4 mSv y-1[7]. External exposures outdoors arise mainly from terrestrial radionuclides. The specific levels are related to the types of rock from which the soils originate. Higher radiation levels are associated with igneous rocks, such as granite, and lower levels with sedimentary rocks. There are exceptions, however, shale and phosphate rocks have relatively high content of radionuclides.

2. Experimental Technique

2.1.Sampling and Sample Preparation

Twenty-six sludge samples were collected from (Nine Stations) in El-Mynia, governorate is shown in Figure 1. All samples, each about 300 gm in weight, were dried in an oven at about 110 C° to ensure that moisture is completely removed, the samples were crushed, homogenized, and sieved through a 200 μ m, which is the optimum size enriched in heavy minerals. Weighted samples were placed in polyethylene beaker, of 350 cm³ volume each. The beakers were completely sealed for 4 weeks to reach secular equilibrium when the rate of decay of the daughters becomes equal to that the parent this step is necessary to ensure that radon gas is confined within the volume and the daughters will remain in the sample [8].

2.2.Instrumentation and Calibration

Radioactivity measurements were performed by gamma ray spectrometer, employing a scintillation detector $3'' \times 3''$. Its hermetically sealed assembly which includes a high-resolution NaI (Tl) crystal, photomultiplier tube, an internal magnetic/light shield, aluminum housing and a 14 pin connector coupled to PC-MCA Canberra Accuspes. It has the following

specifications: (1) resolution 7.5% specified at the 662 keV peak of 137 Cs, (2) window aluminum 0.5 mm thick, density147 mg/cm², (3) reflector oxide; 1.6 mm thick; density 88 mg/cm²,(4) magnetic/light shield-conetic lined steel and (5) operating voltage positive 900 V (dc).

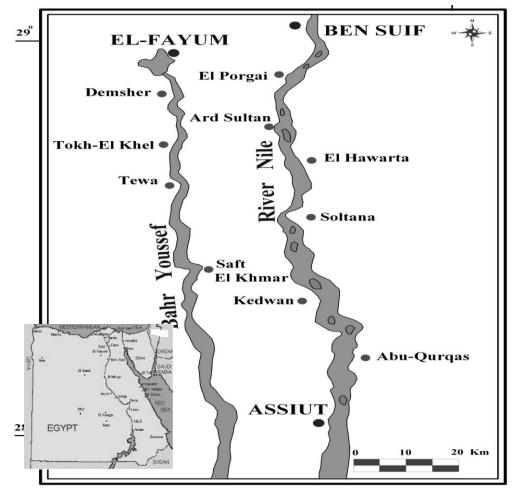


Figure 1. Location Map of the Studied Samples in El-Mynia, Governorate, Egypt

To reduce gamma ray background, a cylindrical lead shield with a fixed bottom and movable cover shielded the detector. The lead shield contained an inner concentric cylinder of copper (0.3 mm thick). The soft component of cosmic rays, consisting of photons and electrons is reduced to a very low level by 100 mm of lead shielding. The X-ray (73.9 keV) emitted from lead by its interaction with external radiation is suppressed by the copper layer [8]. The detection array was energy calibrated using Co-60 (1173.2 and 1332.5 keV), Ba-133(356.1 keV), Cs-137 (661.9 keV) and Ra-226 (1764.49 keV). The efficiency calibration curve was made using different energy peaks covering the range up to 2000 keV. Daily efficiency and energy calibrations for each sample measurement were carried out to maintain the quality of the measurements. In order to determine the background distribution in the environment around the detector, an empty sealed beaker was counted in the same manner

and in the same geometry as the samples. The measurement time of activity or background was 43 200 s. The background spectra were used to correct the net peak area of gamma rays of measured isotopes. The offline analysis of each measured g-ray spectrum has been carried out by a dedicated software program Genie 2000 [9].

The ²²⁶Ra radionuclide was estimated from the 609.3 keV (46.1%) γ -peak of ²¹⁴Bi, 351.9 keV (36.7%), 1120.3 keV (15%), 1728.6 keV (3.05%) and 1764 keV (15.9%) γ -peak of ²¹⁴Pb. The 186 keV photon peak of ²²⁶Ra was not used because of the interfering peak of ²³⁵U with energy of 185.7 keV. ²³²Th radionuclide was estimated from the 911.2 keV (29%) γ -peak of ²²⁸Ac and 238.6 keV (43.6%) γ -peak of ²¹²Pb. ⁴⁰K radionuclide was estimated using 1,461 keV (10.7%) γ -peak from ⁴⁰K itself. The below detectable limit (BDL) were 25.2 Bq kg⁻¹ for ⁴⁰K, 6.5 Bq kg⁻¹ for ²²⁶Ra and 5.7 Bq kg⁻¹ for ²³²Th. All procedures were described in previous publication [10].

3. Chemical Analysis of the Studied Samples

Nine samples were chemically analyzed from nine drinking water purification stations (Kedwan, Elhawarta, Demsher, Tewa, Ard Sultan, Tokh Elkhel, Abu Flow, Saft Elkhmar and Zohrt Elporgi), using scanning electron microscopy (SEM) technique at electron microscope unit, Assiut university, Egypt. The chemical compositions (wt. %) of the studied samples were listed in table (1). The Underground water Saft Elhkamar station has the highest amount of iron (Fe) and manganese (Mn) element; the surface water Abu Flow station has the highest amount of zinc (Zn) and cadmium (Cd) element. Figures (2 to 4) show the results of table (1) in graphical form.

Element									a b
	Kedwan	Elhawarta	Demsher	Tewa	Ard	Tokh	Abu	Zohrt	Saft
					Sultan	Elkhel	Flow	Elporgi	Elkhmar
Al	6.31	6.52	3.09	13.17	24.02	3.39	2.45	1.65	0.09
Si	48.24	53.91	48.79	43.99	47.34	57.84	24.78	46.24	30.35
Р	6.52	7.47	9.27	2.11	2.84	2.72	2.41	5.61	3.51
S	1.11	0.83	0.35	0.57	0.95	0.31	0.3	0.44	0.61
K	1.68	1.64	2.06	1.45	1	1.35	1.76	1.54	0.55
Ca	10.51	5.89	6.02	5.46	4.14	15.49	8.95	8.01	7.28
Ti	2.05	1.81	1	2.19	1.22	1.63	2.31	2.01	0.25
Mn	1.29	1.64	1.45	1.13	0.66	0.61	1.34	0.48	23.52
Fe	20.3	19.13	23.08	26.99	16.23	15.32	48.6	29.65	32.94
Со	0.01	0.34	2.07	0.03	0.01	0.01	1.48	0.41	0.06
Ni	0.24	0.24	1.72	0.57	0.57	0.41	0.29	0.87	0.05
Zn	0.92	0.14	0.01	2.14	0.63	0.97	4.16	2.59	0.53
Cd	0.82	0.43	1.08	0.2	0.39	0.05	1.18	0.54	0.28
Total %	100	100	100	100	100	100	100	100	100

Table 1. Chemical Composition (wt. %) of the Studied Samples

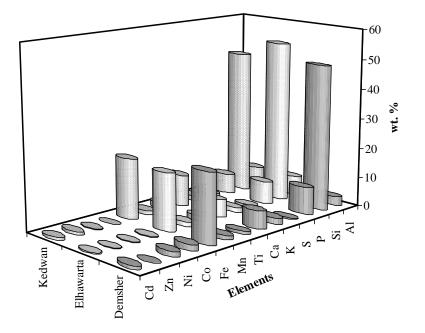


Figure 2. Chemical Analysis for Kedwan, Elhawarta and Demsher Stations

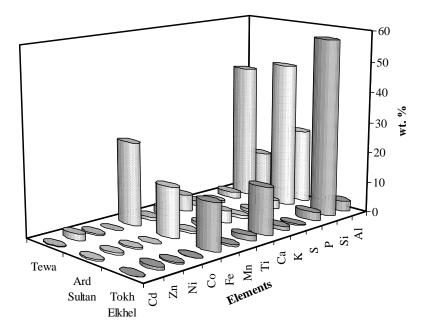


Figure 3. Chemical Analysis for Tewa, Ars Sultan and Tokh Elkhel Stations

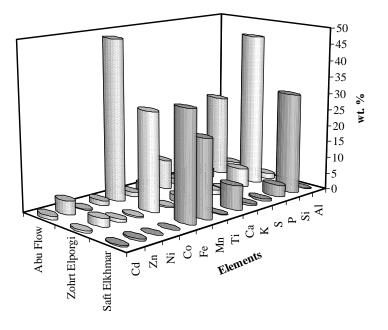


Figure 4. Chemical Analysis for Abu Flow, Zohrat Elporrgi and Saft Elkhmar Stations

4. Results and discussion

It is known that the radioactivity in sediments is similar to that in rocks, usually bed rocks, from which it derives. In this study, the concentration the various ²²⁶Ra, ²³²Th and ⁴⁰K activity concentrations and their corresponding total uncertainties for the 26 sediment samples under investigation were listed in Table 2. The range of specific activity for ²²⁶Ra, ²³²Th and ⁴⁰K were found to be from 6±0.7 to 188±9.4, from 5±1 to 117±5.8 and from 47±4.5 to 412±20.6 Bq Kg⁻¹ respectively. It was clearly evident that ⁴⁰K always contributed to the most specific activity compared with ²³²Th and ²²⁶Ra. The Tokh Elkhel drinking water purification stations samples presented the lowest activity concentrations its averages are 13±0.7 and 119±6 Bq kg⁻¹ for ²²⁶Ra and ⁴⁰K, respectively. While the Abu Flow drinking water purification stations sediment samples presented the lowest activity concentrations are with averages 6±0.6 Bq kg⁻¹ for ²³²Th. The Kedwan drinking water purification station sediment samples presented the highest activity concentrations are with averages 76±4.2 and 86±4.25 Bq kg⁻¹ for ²²⁶Ra and ²³²Th respectively. While the Zohrt Elporgi drinking water purification stations samples presented the highest activity concentrations are with averages 283±15 Bq Kg⁻¹ for ⁴⁰K. Figure 5 shows the results in Table 2 in graphical form.

Figure 6 shows the correlation between the concentrations of the two radioactive isotopes 40 K and 226 Ra in samples under investigation (correlation coefficient =0.51). The Figure 7 indicates that there is a negative correlation (correlation coefficient =0.71) was clearly found between the 226 Ra concentration (Bq kg⁻¹) and Si (wt).While a positive good correlation coefficient (0.89) was found for the relation between 226 Ra concentration (Bq kg⁻¹) and Fe (wt) as it appears in Figure 8.

From the rest Figures 9, 10 and 11, it is clear that there were positive low correlations between (226 Ra and Cd), (40 K and Cd) and (226 Ra and Zn) with correlation coefficient 0.44, 0.52 and 0.69, respectively.

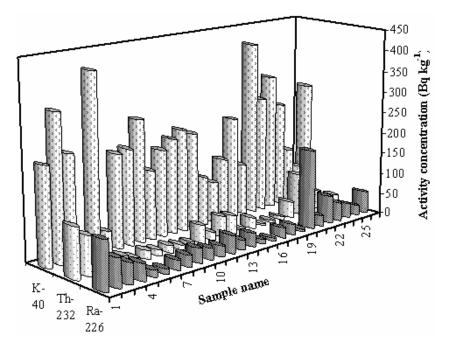


Figure 5. Activity	Concentrations	of 226Ra.	232Th and 4	OK in Different	Samples

Location	Station	Sample	²²⁶ R a	²³² Th	⁴⁰ K
		No.	Bq Kg ⁻¹	Bq Kg ⁻¹	Bq Kg ⁻¹
		1	113±6.7	117 ± 5.8	227±11.4
Kedwan	Surface water	2	67±3.4	91±4.5	337±16.8
		3	49 ± 2.5	50 ± 2.5	241±12.1
mean			76±4.2	86±4.25	268±13.4
		4	40±3.1	12.5±0.6	47±6.9
Elhawarta	Surface water	5	17 ± 1.9	23±1.2	412±20.6
		6	12 ± 0.8	11±0.6	47±4.5
mean			23±1.9	15.5±0.8	168±10.6
		7	24±2.3	20±3	218±13
Demsher	Surface water	8	27±3.5	14±3.2	223±13.3
		9	34±1.7	17±0.9	285±14.2
mean			28±2.5	17±2.3	242±13.5
		10	28±2.6	11±1.4	162±9.3
Tewa	Surface water	11	25±1.3	10 ± 0.6	204±10.2
		12	42 ± 2.1	42 ± 2.1	224±11.2
mean			31±2	21±1.3	196±10
		13	24 ± 2.7	19±2.3	239±13.5
Ard Sultan	Surface water	14	32±1.9	49±3.7	226±11.3
mean			28±2.2	34±3	232.5±12.4
		15	10 ± 0.5	45±2.2	115±5.8
Tokh Elkhel	Surface water	16	6±0.7	5±1	97±4.8
		17	24±1	25±1.2	147±7.3

Table 2. Activity Concentrations of ²²⁶Ra, ²³²Th and ⁴⁰K in Different Samples

International Journal of Advanced Science and Technology Vol. 42, May, 2012

mean			13±0.7	25±1.4	119±6
		18	24±2.1	5±0.3	240±14.2
Abu Flow	Surface water	19	7±1.4	5±1.3	122 ± 10.1
		20	188 ± 9.4	7±0.3	412±44.5
mean			73±4.3	6±0.6	258±22.9
		21	25±3.6	38±3.1	274±16.6
Zohrt Elporgi	Surface water	22	66±3.3	99±4.9	324±16.2
		23	44±3.2	43±2.1	251±12.6
mean			45±3.3	60±3.3	283±15
		24	34±2.4	45±3.2	132±10.0
Saft Elkhmar	Underground water	25	23±1.4	26±1.3	84±4.2
		26	51±2.5	23±1.1	285±14.2
mean			36±2.1	31±1.8	167±9.5

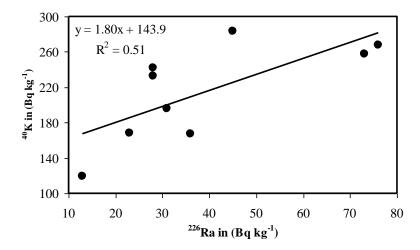


Figure 6. The Correlation between ⁴⁰K and ²²⁶Ra Concentration in Selected Samples

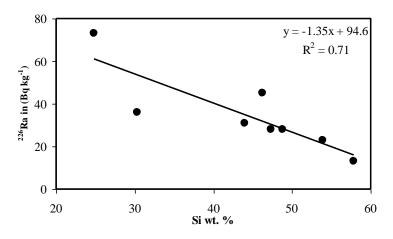


Figure 7. The Relation between ²²⁶Ra concentration (BqKg⁻¹) and Si (wt. %)

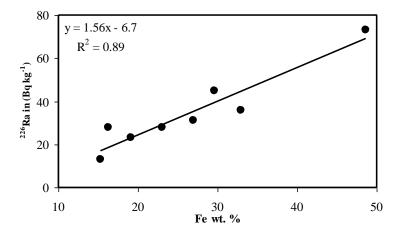


Figure 8. The Relation between ²²⁶Ra Concentration (BqKg⁻¹) and Fe (wt. %)

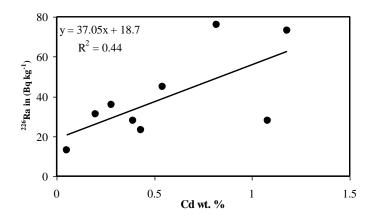


Figure 9. The Relation between ²²⁶Ra Concentration (BqKg⁻¹) and Cd (wt. %)

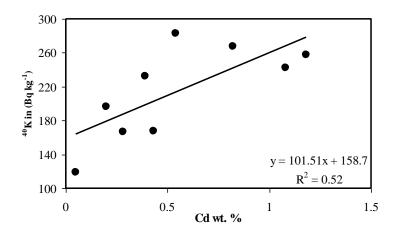


Figure 10. The Relation between ⁴⁰K Concentration (BqKg⁻¹) and Cd (wt. %)

International Journal of Advanced Science and Technology Vol. 42, May, 2012

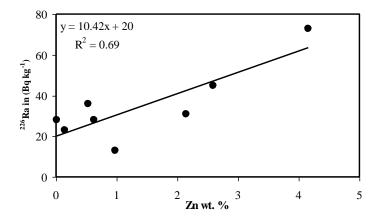


Figure 11. The Relation between ²²⁶Ra Concentration (BqKg⁻¹) and Zn (wt. %)

4.1. Radium Equivalent Activity (Re_{eq})

Radium equivalent concentration (Raeq) is a common index used to compare the specific activities of materials containing ²²⁶Ra, ²³²Th, and ⁴⁰K. It can be expressed as:

$$Ra_{eq} = A_{Ra} + 1.43A_{Th} + 0.007A_K$$
(1)

Where, A_{Ra} , A_{Th} and A_K are specific activities of ²²⁶Ra, ²³²Th and ⁴⁰K, respectively, in Bq kg⁻¹. Radium equivalent concentration (Raeq) was calculated based on the estimation that 370 Bq kg⁻¹ of ²²⁶Ra, 259 Bq kg⁻¹ of ²³²Th and 4,810 Bq kg⁻¹ of ⁴⁰K produce the same g-ray dose rate [11].

The results obtained in Table 3 showed that, the lowest Ra_{eq} was 19.4 Bq kg⁻¹ in samples from Tokh Elkhel station, while the highest value was 296.6 Bq kg⁻¹ in Kedwan station. These values are less than the maximum admissible value of 370 Bqkg⁻¹ [11].

4.2. Absorbed Gamma Dose Rate (D)

The absorbed dose rates due to gamma radiations in air at 1m above the ground surface for the uniform distribution of the naturally occurring radionuclides (226 Ra, 232 Th and 40 K) were calculated based on guidelines provided by UNSCEAR [12]. The conversion factors used to compute absorbed gamma dose rate (D) in air per unit activity concentration in Bq/kg (dry weight) corresponds to0.462nGy/h for 226 Ra, 0.604 nGy/hfor 232 Th and 0.042 nGy/h for 40 K. Therefore D can be calculated as follows [13]:

$$D = 0.462C_{Ra} + 0.604C_{Th} + 0.0417 C_{K}$$
(2)

Where $C_{Ra,} C_{Th \text{ and }} C_K$ are the concentration in $(BqKg^{-1})$ of radium, thorium and potassium respectively. Column 4 of Table 3 gives the results for absorbed dose rate in air for samples under investigation. absorbed gamma dose rate for all stations ranged from 9.8 to 135.6 nGyh⁻¹, where the highest value was 135.6 nGyh⁻¹ in Kedwan station while lowest value was 9.8 nGyh⁻¹ in samples from Tokh Elkhel station.

4.3. Annual Effective Dose

In order to estimate the annual effective doses, one has to take into account the conversion coefficient from absorbed dose in air to effective dose and the indoor occupancy factor. A value of 0.7 Sv Gy⁻¹ was used for the conversion coefficient from absorbed dose in air to effective dose received by adults, and 0.8 for the indoor occupancy factor, implying that 20% of time is spent outdoors, on average, around the world. The annual effective dose rate outdoors in units of μ Svy-1 is calculated by the following formula [14]:

Annual Effective Dose Rate =
$$D \times T \times F$$
 (3)

Where D is the calculated dose rate (in nGy h⁻¹), T is the outdoor occupancy time (0.2x24 h x365.25 d \approx 1753 h y-1), and F is the conversion factor (0.7x10-6 Sv Gy-1). The experimental results of Annual Effective Dose Rate are presented in Table (3), column 6.

4.4. External Hazard Index

The external hazard index (Hex) was determined from the criterion formula as follow [15]

$$H_{ex} = \frac{C_{Ra}}{370} + \frac{C_{Th}}{259} + \frac{C_K}{4810} \le 1$$
(4)

Table 3. Equivalent Radium (Bq/kg), the Dose Rate (nGy/h), External Hazard Indices (Hex), Annual Effective Dose Rate, and Excess Lifetime Cancer Risk (ELCR)

	Sample number	$Ra_{eq} (Bq.kg^{-1})$	Dose rate (nGy.h ⁻¹)	H_{ex} $(nGy.h^{-1})$	Ann. Eff.Dose	ELCR
					μSv/y	×10 ⁻⁴
	1	296.6	135.6	0.8	164.7	5.8
Kedwan	2	220.5	103.2	0.6	125.4	4.4
	3	134.8	64.6	0.4	78.4	2.7
	4	61.7	27.6	0.2	33.5	1.7
Elhawarta	5	78.9	40.3	0.2	48.9	1.4
	6	31.6	14.7	0.1	17.8	6.2
	7	67.5	32.7	0.2	39.7	1.4
Demsher	8	61.6	29.9	0.2	36.3	1.3
	9	78.4	38.1	0.2	46.3	1.6
	10	54.6	26	0.2	31.6	1.1
Tewa	11	54.7	26.6	0.2	32.3	1.1
	12	117.9	55.4	0.3	67.3	2.4
	13	68.7	33.5	0.2	40.7	1.4
Ard Sultan	14	117.4	55.6	0.3	67.5	2.4
	15	82.3	39	0.2	47.3	1.7
Tokh Elkhel	16	19.4	9.8	0.1	11.9	4.2
	17	69.8	33.6	0.2	40.1	1.4
	21	107.9	50.1	0.3	60.9	2.1
Zohrt Elporgi	22	65.8	30.5	0.2	37.1	1.3
	23	103.1	48.9	0.3	59.4	2.1
	24	99.4	48	0.3	58.3	2.0
Soft Elkhmar	25	230	107.5	0.6	130.6	4.6
	26	123.1	58.1	0.3	70.5	2.5

Where, C_{Ra} , C_{Th} and C_K are activities of ²²⁶Ra, ²³²Th, and ⁴⁰K, respectively, in Bq kg⁻¹. To limit the external gamma radiation dose from drinking water purification stations to 1.5 mSv year⁻¹ external hazard index must be less than unity in order to maintain the radiation hazard negligible [15]. The external hazard index shows in Table 3.

Excess lifetime cancer risk (ELCR) was calculated as shown in column (7) in Table 3 by using the following equation:

 $ELCR = AEDE \times DL \times RF$ (5)

Where DL is duration of life (70 year) and RF is risk factor (Sv^{-1}) fatal cancer risk per Sievert. For stochastic effects, ICRP 60 [13] uses values of 0.05 for the public. The highest value (5.8E-04) for samples from Kedwan station whiles the lowest one (1.1E-05) for samples from Tewa station.

5. Conclusion

The present work shows that the natural radioactivity levels in the sediment samples which were collected from nine drinking water purification stations namely Kedwan, Elhawarta, Demsher, Tewa, Ard Sultan, Tokh Elkhel, Abu Flow, Saft Elkhmar and Zohrt Elporgi in El-Mynia, Egypt (in general) are well below the acceptable limits. The average values of all the calculated radiological indices (absorbed dose rate, annual effective dose, radium equivalent and hazard index) extracted from this activity, in all investigated samples are within the levels recommended by Radiation Protection 112 (EC, 1999) and UNSCEAR 2000 report. Therefore, the sediments do not pose any significant radiological threat to the population.

Acknowledgment

This work was carried out using the nuclear analytical facilities at Physics Department-Faculty of Sciences – Al-Azhar University – Assiut, Egypt

References

- [1] M. Degerlier and G. Karahan, "Natural radioactivity in various surface waters in Adana, Turkey", Desalination, vol. 261, (2010), pp. 126.
- [2] M. Palomo, A. Peñalver, C. Aguilar and F. Borrull, "Presence of Naturally Occurring Radioactive Materials in sludge samples from several Spanish water treatment plants", Journal of Hazardous Materials, vol. 181, no. 1-3, (2010), pp. 716.
- [3] K. Ross and A. Riaz, "Naturally occurring radionuclides in materials derived from urban water treatment plants in southeast Queensland, Australia", Journal of Environmental Radioactivity, vol. 99, (2008), pp. 607.
- [4] A. Aksoy, M. Ahmed, W. S. A. Matterm and Z. R. El-Naggar, "Gamma-ray spectroscopic and PIXE analysis of selected samples from the phosphorite deposits of Northwestern Saudi Arabia", Radioanal. Nucl. Chem., vol. 253, no. 3, (2002), pp. 517.
- [5] R. J. Budnitz, A. V. Nero, D. J. Murphy and Graven, R. Instrum. Environ. Monit., vol. 1, (1983), pp. 433.
- [6] J. De Oliveria, B. Paci Mazzilla, P. da costa and A. Tanigava, "Natural Radioactivity in Brazilian bottled mineral waters and consequent doses", J. radioanal. Nucl. Chem., vol. 249, no. 1, (2001), pp. 173.
- [7] United Nations Scientific Committee on the Effect of Atomic Radiation, Sources and effects ionizing radiation, Report No. D, 420(PA: ASTM), (1986), pp. 109.
- [8] A. G. E. Abbady, M. A. M. Uosif, A. El-Taher, "Natural radioactivity and dose assessment forphosphate rocks from Wadi El-Mashash and El-Mahamid Mines", Egypt, vol. 48, no. 1, (2005), pp. 65.
- [9] GENIE-2000 Basic Spectroscopy (Standalone) V1.2A Copyright (c), Canberra Industries, (1997).
- [10] M. A. M. Uosif, "Gamma-ray spectroscopic analysis of selected samples from Nile river sediments in Upper Egypt", Radiat. Prot. Dosim, vol. 123, (2007), pp. 215.

- [11] M. A. M. Uosif and L. M. Abdel-Salam, "An assessment of the external radiological impact in granites and pegmatite in central eastern desert in Egypt with elevated natural radioactivity", Radiat. Prot. Dosim., vol. 147, no. 3, (2011), pp. 467.
- [12] UNSCEAR, United Nations Scientific Committee on the Effects of Atomic Radiation, Sources, effects and risks of ionizing radiation, Report to the General Assembly with annex B, United Nations, New York, (2000).
- [13] ICRP-60, International Commission on Radiological Protection Recommendations of the International Commission on Radiological Protection. Annals of the ICRP, vol. 21, (1991), pp. 1–3.
- [14] K. Saito, N. Petoussi and M. Zanki, "Calculation of organ doses from environmental gamma rays using human phantoms and Monte Carlo", Part 1. Monoenergetic sources of natural radionuclides in the ground, GSF-B2/90, (1990).
- [15] L. Xinwei, "Natural radioactivity in some building materials and by- ICRP", Recommendations of the International Commission on Radiological Protection, vol. 21, no. 1–3, (**1990**), pp. 60.
- [16] EC, European commission report on radiological protection principles concerning the natural radioactivity of building materials. Radiat. Prot., vol. 112, (1999).

International Journal of Advanced Science and Technology Vol. 42, May, 2012